



**ROSATOM**

STATE ATOMIC ENERGY CORPORATION

# International Cooperation for Development of Innovative Energy Technologies

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JSC Science and Innovations

Rosatom State Corporation



Innovation Management Company “Science and Innovation” was established in 2011 to control the assets and R&D activities of Rosatom research institutes

We possess **the whole range of competences necessary to provide scientific services** in the following fields: energy and non-energy nuclear technologies, construction and functional materials, extraction and processing of raw materials, radiation application and survey technologies

Research and Development

Scientific Services and  
High-tech Manufacturing

Developing Comprehensive  
Scientific Programs for  
Customers

Developing Scientific  
Infrastructure for  
Customers

Training for Scientific  
Personnel

## Department of Energy Physics



## Department of Chemistry and Technology



## Department of Electricity Physics



**11 scientific research institutes**, including:

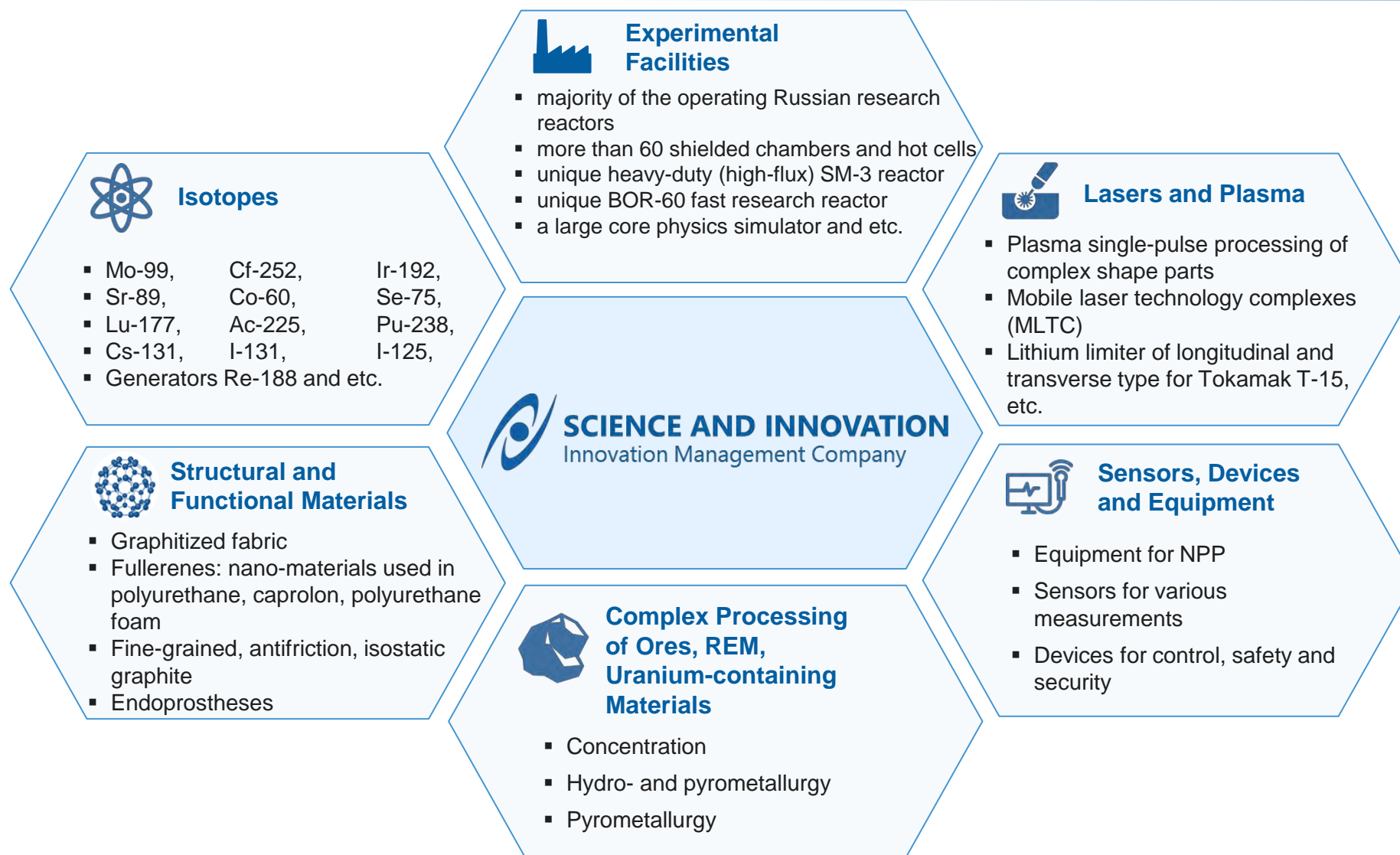
- 3 public science centers (RIAR, IPPE, TRINITI)

**9.5 thousand employees**, including:

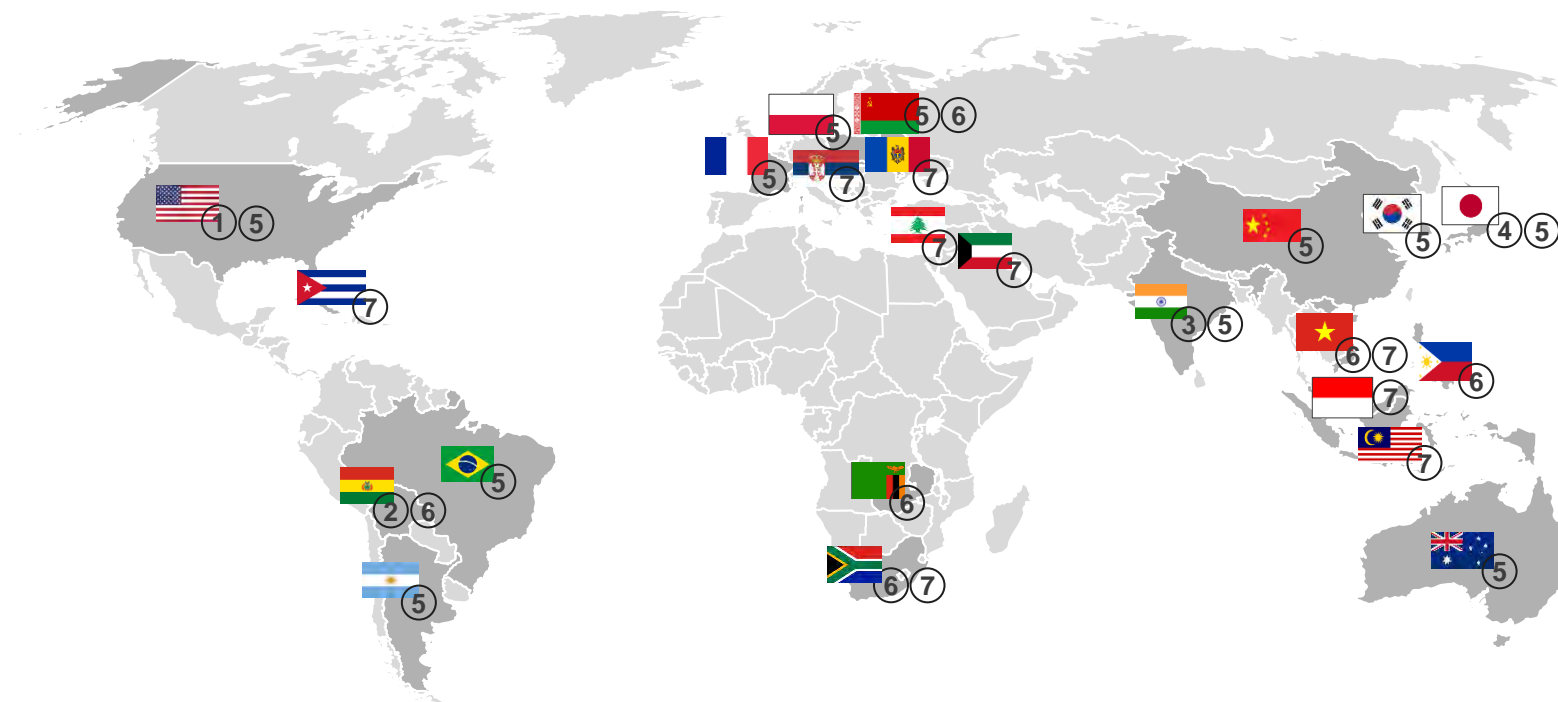
- 1936 researchers
- 154 Doctors of Sciences
- 557 Candidates of Sciences
- 77 postgraduates

**182 experimental research facilities**, including:

- several unique facilities for reactor material science, plasma research and laser technologies



# International Projects of the Scientific Division



**1 USA**  
JSC "IRM" supplies Ir-192 to the largest consumer of this isotope in the world - QSA Global Inc. (USA)

**2 Bolivia**  
Transport radiation monitors and radiation safety system by JSC "Scientific and Technical Center "Yafi" were installed at the El Alto airport

**3 India**  
Sources of radiation based on curium-244 produced by SSC SRIAR JSC were delivered to the Laboratory of Physical Investigations

**4 Japan**  
JSC IPPE developed a neutron detector for the study of the Fukushima NPP

**5 Australia, Argentina, Belarus, Brazil, India, China, Poland, United States, France, Republic of Korea, Japan**  
Supply of isotope products is carried out by JSC Isotope

**6 Belarus, Bolivia, Vietnam, the Philippines, South Africa, Zambia**  
Bolivia is creating a Center for Nuclear Science and Technology (CNST), other countries are considering a CNST

**7 Vietnam, Indonesia, Cuba, Kuwait, Lebanon, Malaysia, Moldova, Serbia, South Africa**  
Radiation portal monitors and united safety radiation system were developed by JSC "Scientific and Technical Center" Yafi" jointly with the IAEA

# International Cooperation: Key Facts 2018



## Commercial cooperation

- R&D services
- Innovative production
- Isotopes
- CNST
- Lasers and plasma

**30**  
countries

**56**  
products and  
services

**188**  
international events

**58**  
receptions of  
foreign delegations

**\$48M**  
2018 revenue

**\$111M**  
2018 portfolio of orders

**\$400M**  
2019 potential  
portfolio of orders

## Non-commercial cooperation

- Scientific and technical cooperation
- IAEA conferences
- Technical tours
- Exchange programs

# Two-component Atomic Energy - a Synergistic Coexistence of a Reactor park on Thermal and Fast Neutrons



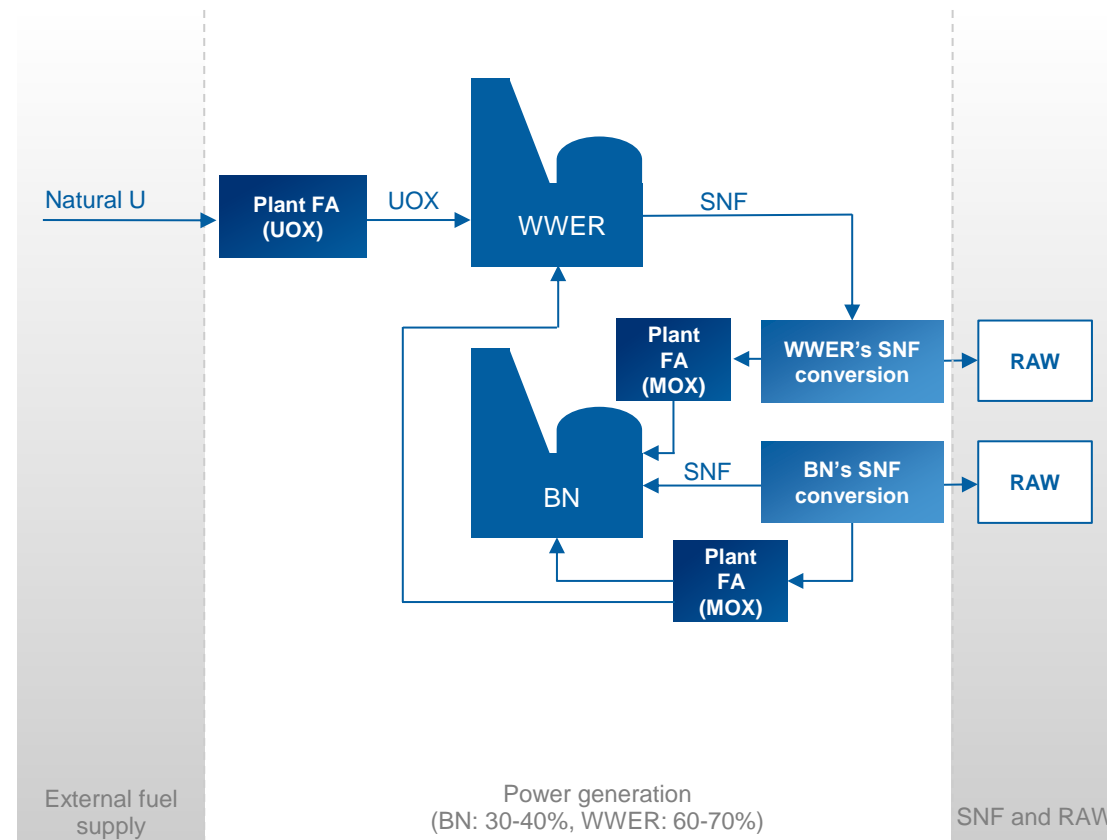
Reactors on  
thermal neutrons

+



Reactors on  
fast neutrons

- Solving the problem of SNF accumulation in the world
- x10 increase in efficiency of using raw uranium
- Minimize the waste with a half-life reduction due to minor actinides transmutation in fast neutron reactors



**Two-component atomic energy is also based on a cheaper WWER technology**

**Recycling of nuclear fuel requires effective solutions to improve SNF reprocessing technologies and to include minor actinides in the fuel composition**

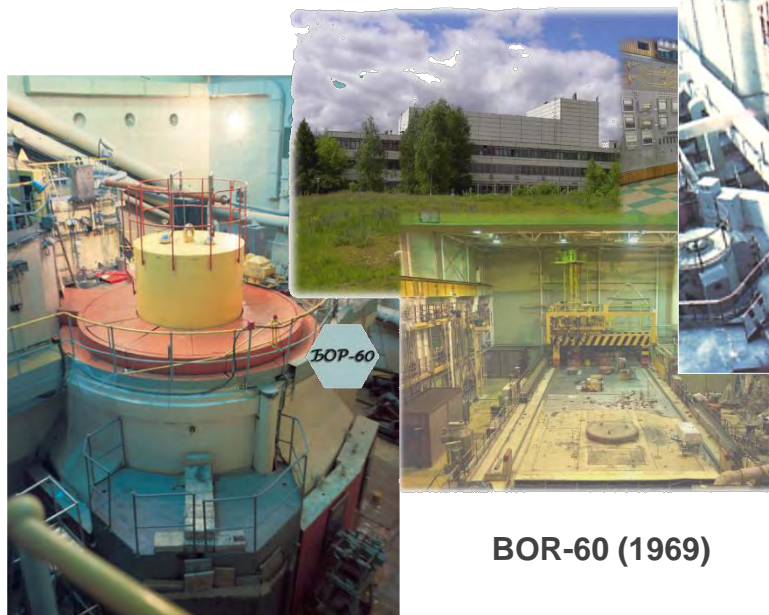


# Sodium Fast Reactor Development Stages in Russia



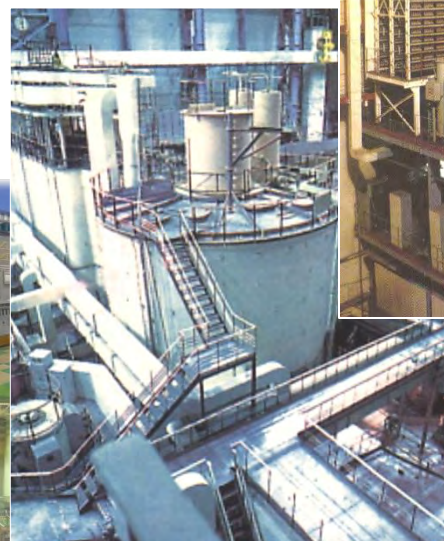
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Rosatom Scientific Division is the Scientific Leader of the Russian Sodium Fast Reactors



BR-5/10 (1959)

BOR-60 (1969)



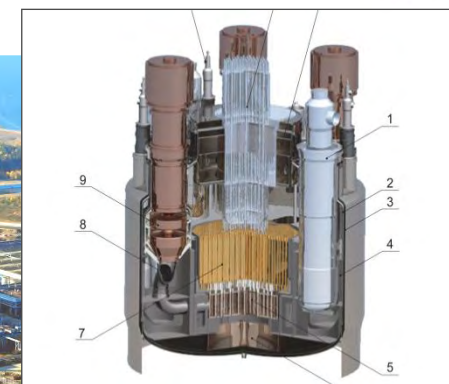
BN-350 (1973)



BN-600 (1980)



BN-800 (2015)



BN-1200 (2027)

- The main peculiarity of the Fast Reactors is that  $^{238}\text{U}$  and  $^{232}\text{Th}$  could be involved into the Nuclear Fuel Cycle. Natural reserves of the  $^{238}\text{U}$  and  $^{232}\text{Th}$  are much bigger than  $^{235}\text{U}$ , which now is the main fuel for the Thermal Neutron Reactors.
- Fast Reactors could solve the Spent Nuclear Fuel issue of the currently existing Thermal Neutron Reactors.
- These features have the most value in Closing of the Nuclear Fuel Cycle

# Fast Neutron Critical Facilities and Research Reactors

## Rosatom Scientific Division is the Scientific Leader of the Russian Sodium Fast Reactors

### IPPE (Obninsk):

- BR-1 (1955)
- BR-2 (1956)
- BR-5 (1959)
- BR-10 (1973)
- BFS-1 (1961)
- BFS-2 (1970)

### RIAR (Dimitrovgrad):

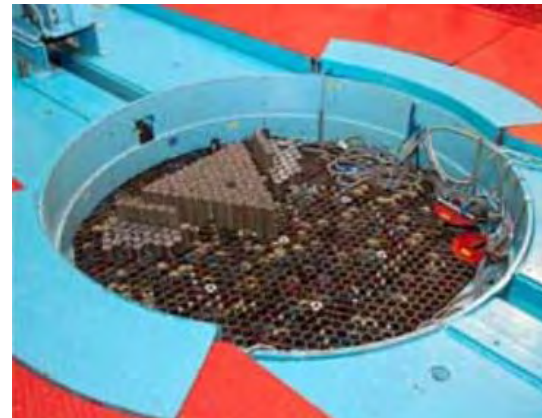
- BOR-60 (1969)
- MBIR (to be commissioned in 2025)



BR-1 – the First European Fast Neutron Research Reactor



BFS-2 – The World Biggest Experimental Facility



BFS-1 – Benchmark Experimental Complex



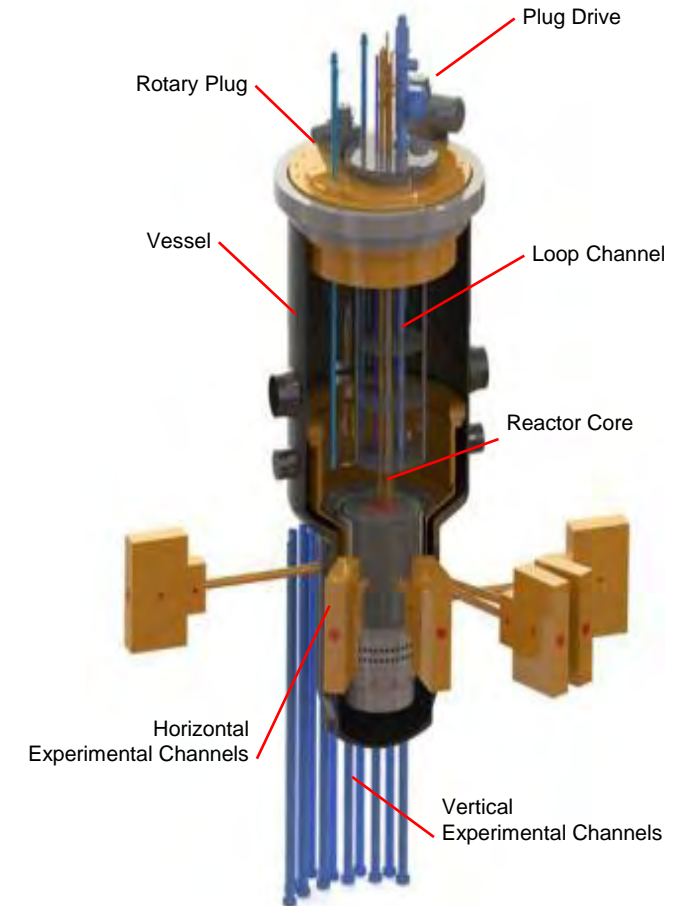
MBIR Multifunctional Research Reactor (model)



# MBIR – a Unique Multipurpose Research Infrastructure

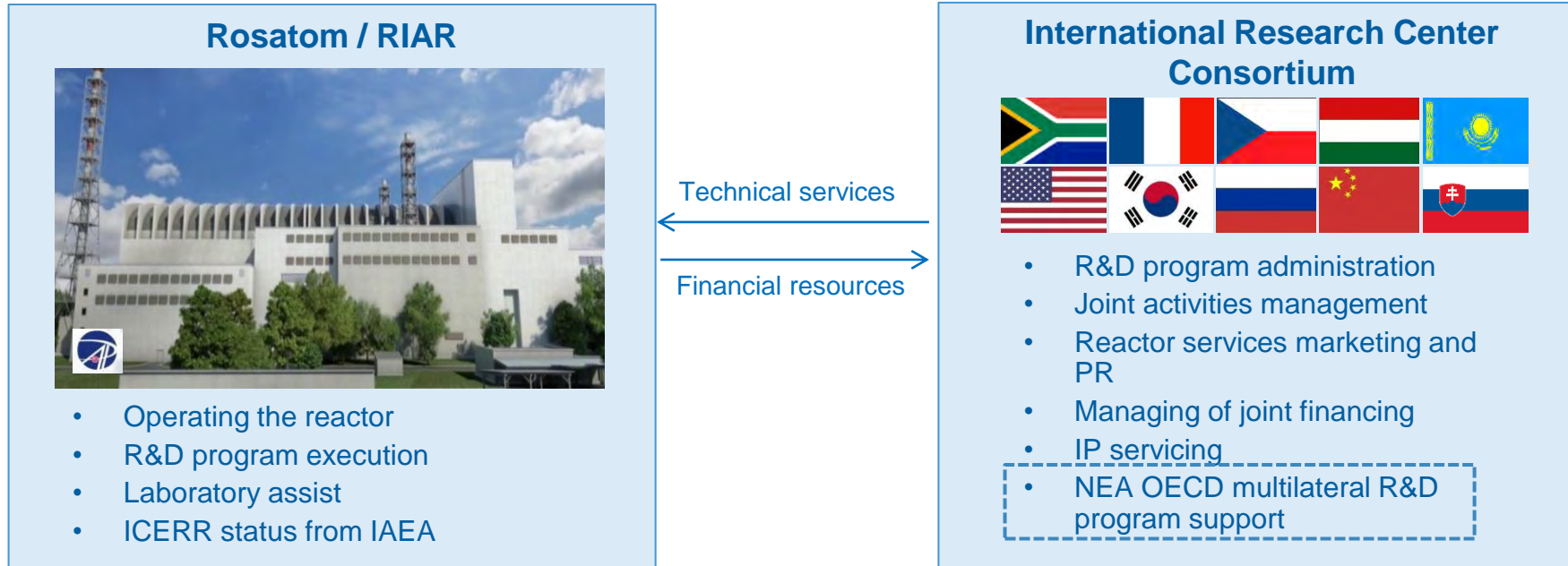
- Fast neutron spectrum, sodium coolant, MOX fuel, 150 MW(t) heating power, 50 MW(t) electric – reactor MBIR will produce electricity for the local market
- Linear heat rate - max. 500 W/cm, max temp on the cladding 700C
- Min designed life time - 50 years
- Upgradeable experimental capabilities, sockets for experimental devices to be tested in conditions close to real
- Under construction at RIAR - Research Institute of Atomic Reactors, a Rosatom enterprise

Main Technical Capabilities	BOR-60	MBIR
Max. neutron flux, 10 <sup>15</sup> sec <sup>-1</sup> cm <sup>-2</sup>	2.8	5.3
MTAs in the core	12	14
Instrumented cells	1	3
Outside loop channels	0	3
Vertical and horizontal channels	12	18
Max. dpa rate in central channel (annual)	22	34
Total dpa/l per annum available in the reactor core	300	1350



Partners can sign a separate service contract in respect to post irradiation studies. RIAR research laboratory complex may perform a wide range of post irradiation research on-site removing the implication of the irradiated materials transportation and RAW disposal

# Current International Research Center Consortium Status



## Advantages of International Cooperation

### For foreign partners:

- Guaranteed opportunity to conduct research at MBIR after 2020
- Sharing experience and knowledge on fast reactors and reactor experiments
- Platform for experimenting and R&D of national projects of innovative reactors

### For global community:

- Possibility of attracting specialists from developing countries to the sphere of fast neutron reactors without any risks for the non-proliferation regime

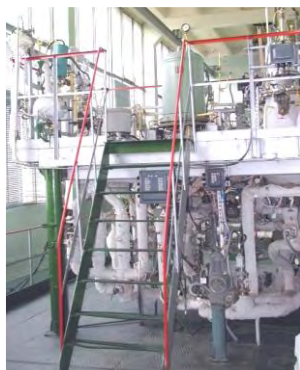
### For Russia:

- Attracting additional experience and knowledge at the development stage of the project
- Expansion of interaction between Russian and foreign specialists
- Additional investments for the project

# Lead Fast Reactor Development Stages in Russia



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**Pb-Bi stand (1951)**



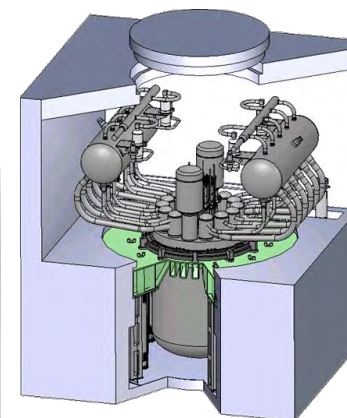
**Test  
Submarine  
'Project  
645' (1963)**



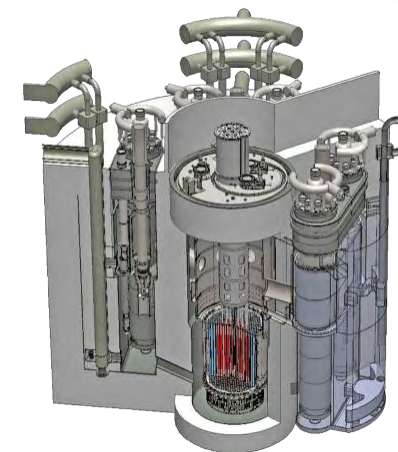
**Test  
Submarine  
'Project 705'  
(1971)**



**Serial  
Submarine  
'Project 705'  
(1976-1996)**

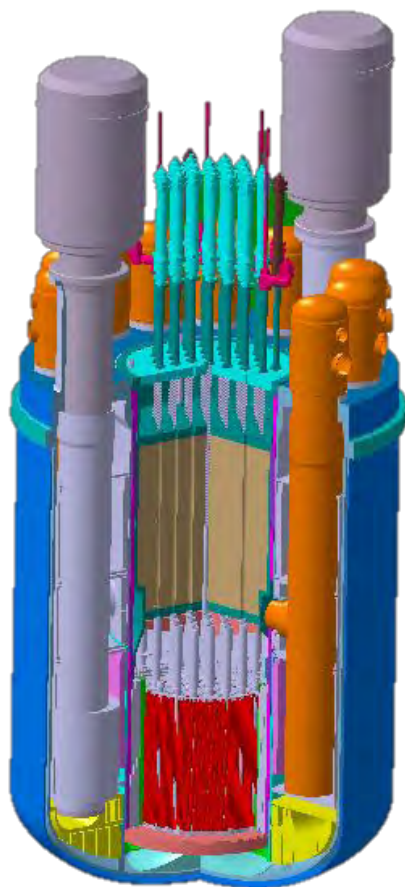


**SVBR-100  
(after 2025)**



**BREST-OD-300  
(after 2025)**

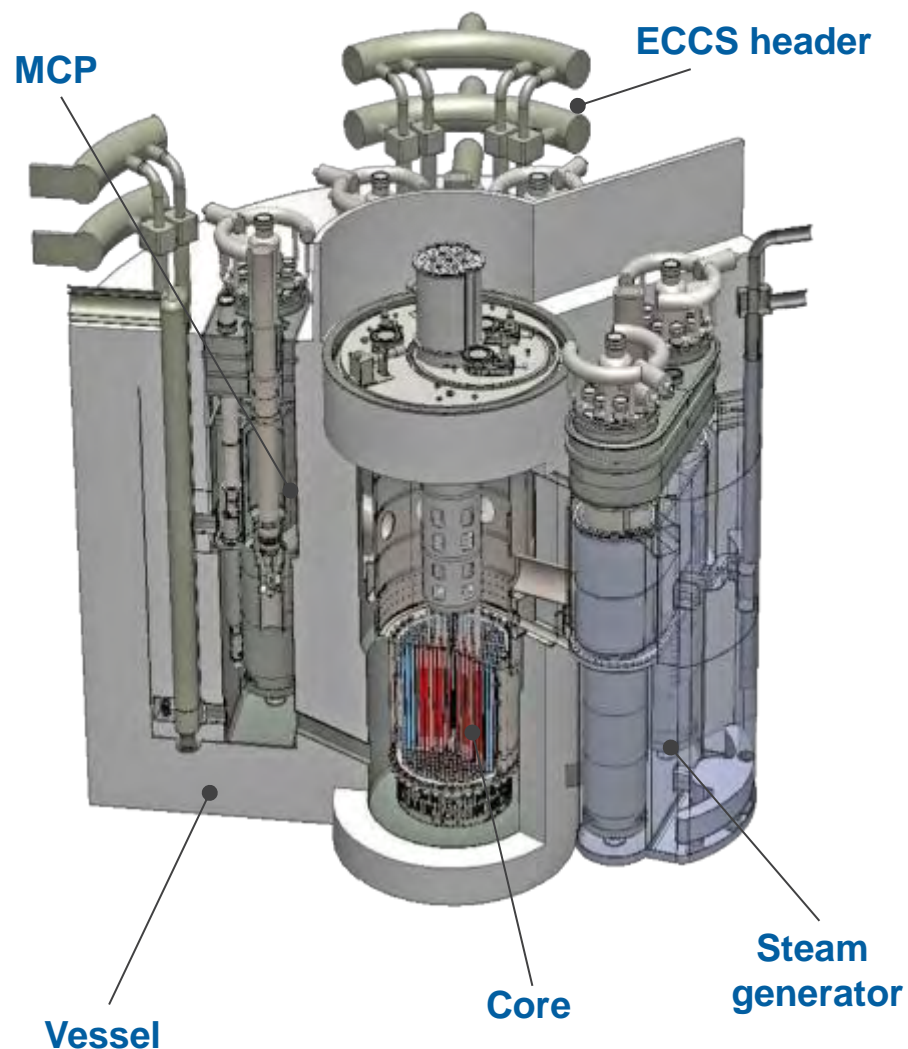
# SVBR-100 Reactor



Parameter	Value
Thermal power (rated), MW	280
Temperature, °C	
at the core outlet	482
at the core inlet	320
Number of fuel rods, items	12114
Average volume power deposition density of the core, kW/dm <sup>3</sup>	140
Average linear loading per fuel rod, kW/m	~243
Fuel type	UO <sub>2</sub>
Loading of U-235, kg	1470
Average enrichment of fuel, at.%	16.1
Volume of coolant in primary circuit, m <sup>3</sup>	18
Dimensions of reactor module: diameter by height, m	4.53×7.55



# BREST-OD-300 Reactor. Scheduled Start-up - after 2021



## BREST-OD-300: innovative technology solutions

- a new coolant type – chemically inert liquid Pb
- equilibrium core with (U- Pu) nitride fuel:
  - low reactivity margin ( $< \beta_{\text{эф}}$ ) at the power level that rules out any accident of the Chernobyl type;
  - full fuel breeding (BRC within the range of 1.04 – 1.06) without weapons-grade Pu production (no blankets);
  - starting with the third year of operation – fuel self-sufficient operation in the closed nuclear fuel cycle;
  - possible use (burning) of long-lived MA;
- elimination of coolant loss, a new ECCS type – an accident of “Fukushima” type is impossible

Thermal power, MW	700
Electrical power, MW	300
Steam production capacity t/h, no less	1480
Primary coolant, volume, m3	Lead 1000
Gas pressure above the LC level: <ul style="list-style-type: none"><li>- overpressure, MPa</li><li>- maximum pressure, MPa</li></ul>	0.003-0.005 0,02
Average Pb temperature at the core inlet/outlet, °C	420/540
Average Pb temperature at the steam generator inlet/outlet, °C	340/505
Number of loops	4



# Prospects of Nuclear Heating

## Currently in Russia

In the Russian climatic conditions heat supply is the most fuel-intensive sector of the economy (more than 40% of the fuel and energy resources).

75% of the consumption of heat energy falls on facilities covered by district heating systems

The use of fossil fuels in heat power has obvious drawbacks:

- **Growing cost** of organic fuel
- **Difficulties in delivering fuel** to remote areas
- **Ecological problems**



## Conditions for the effectiveness of nuclear heating systems

- **lowest cost** of installations
- operation of a nuclear power source mainly in the “**basic mode**”
- **simplicity** of manufacturing, installation and construction of equipment
- **minimum time** for design and construction
- **highest safety** - possibility of placing the installation near the consumer
- **easy maintenance** - no need for a large number of personnel

For effectiveness of heat supply systems of the housing and communal services, **special nuclear power installations needed**

**Under these conditions, nuclear power may prove to be the most effective source of thermal energy for the needs of housing and communal services**

**Russia has obtained successful experience of using nuclear power units for heating**



**The first NPP in the world with AM reactor (Obninsk)**



**The first nuclear heating plant (Bilibino)**

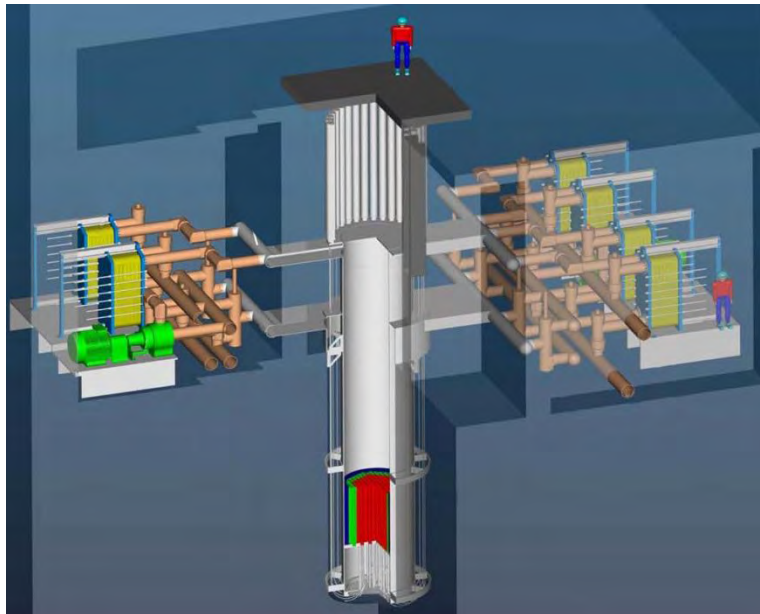


**Beloyarsk NPP with BN-600 fast neutron reactor (Zarechnyi)**

## Reactor Unit for The heat supply with Atmospheric pressure in the primary circuit

Pool reactors do not have excess water pressure in the primary circuit and the reactor pool, which completely excludes the possibility of a number of dangerous emergencies damaging nuclear fuel

Due to high safety indicators, stations with such reactors can be located within inhabited areas and town near the heat consumers



### Advantages

**single-purpose use** – heating supply



**simple design**

(pool reactor and

low pressure and temperature of primary circuit heat carrier)



**reliability and safety**

(a three-circuit scheme for transferring heat to the consumer, a large accumulating water deposit in the pool, feedback reactivity effects and natural circulation in emergency conditions)

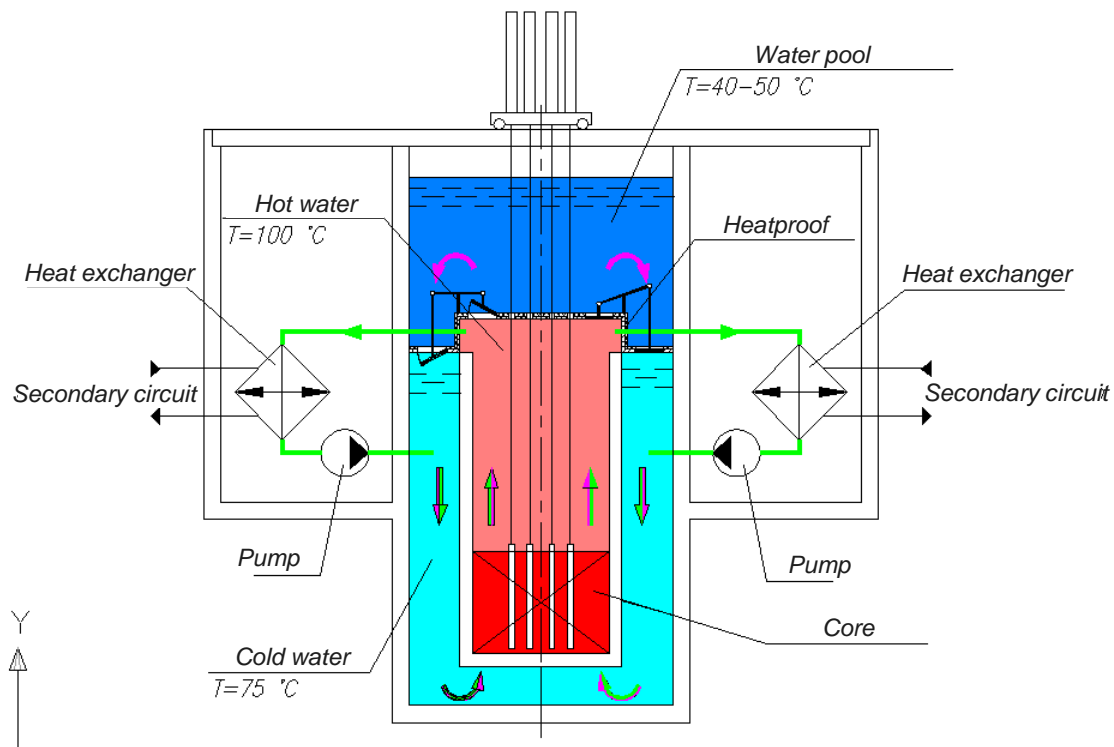


**low cost**

- **Serially manufactured nuclear and non-nuclear equipment is used**
- Nuclear fuel based on low-enriched uranium dioxide **similar to that for VVER-type reactors**
- **Integrated offer:** possibility to implement a **full cycle of operation and maintenance of RUTA reactors**, including service and SNF management

# RUTA Technical Specifications

Thermal power of the reactor	20 – 70 MW
Fuel	UO <sub>2</sub> / cermet
U-235 enrichment, %	3.6 / 5.4
Fuel life cycle	9 (30) years
Period between refueling	3 (10) years
Core temperature (input/output)	75 / 101 °C
Amount of circuits	2
Annual heat supply (Capacity factor=0.66)	100 – 350 thousand Gcal
Life cycle	60 (100) years

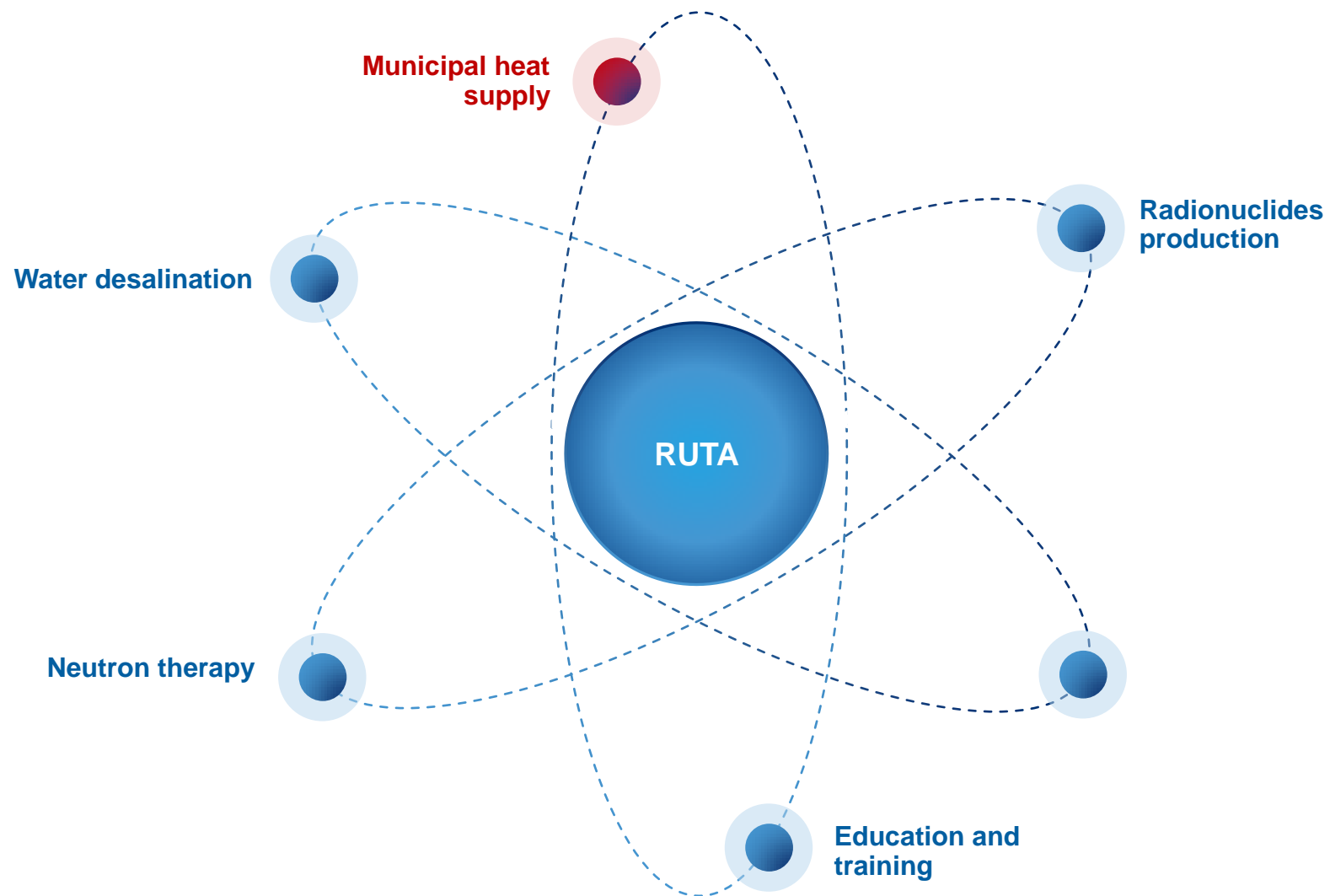


Principal hydraulic scheme of the primary circuit

## Use of RUTA can be extended



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# Japanese Hydrogen Plans



**Japan** – one of the leading countries in the field of hydrogen and hydrogen technologies development.

**Japan** aims to build "**hydrogen economy**" and realize "**hydrogen society**" by 2050.

**Japanese** hydrogen plans up to **10 mln t** in **2050**.

**"Basic Hydrogen Strategy"** - step  
by step demonstration approach to local  
hydrogen market development

- **SUPPLY CHAINS COMMERCIAL STAGE**
- **DEVELOPMENT OF DOMESTIC P2G TECHNOLOGIES**



**PRESENT**

**2030**

**TARGET**

**SUPPLY CHAIN DEVELOPMENT AND  
DEMONSTRATION, SCALE-UP**

Applications:



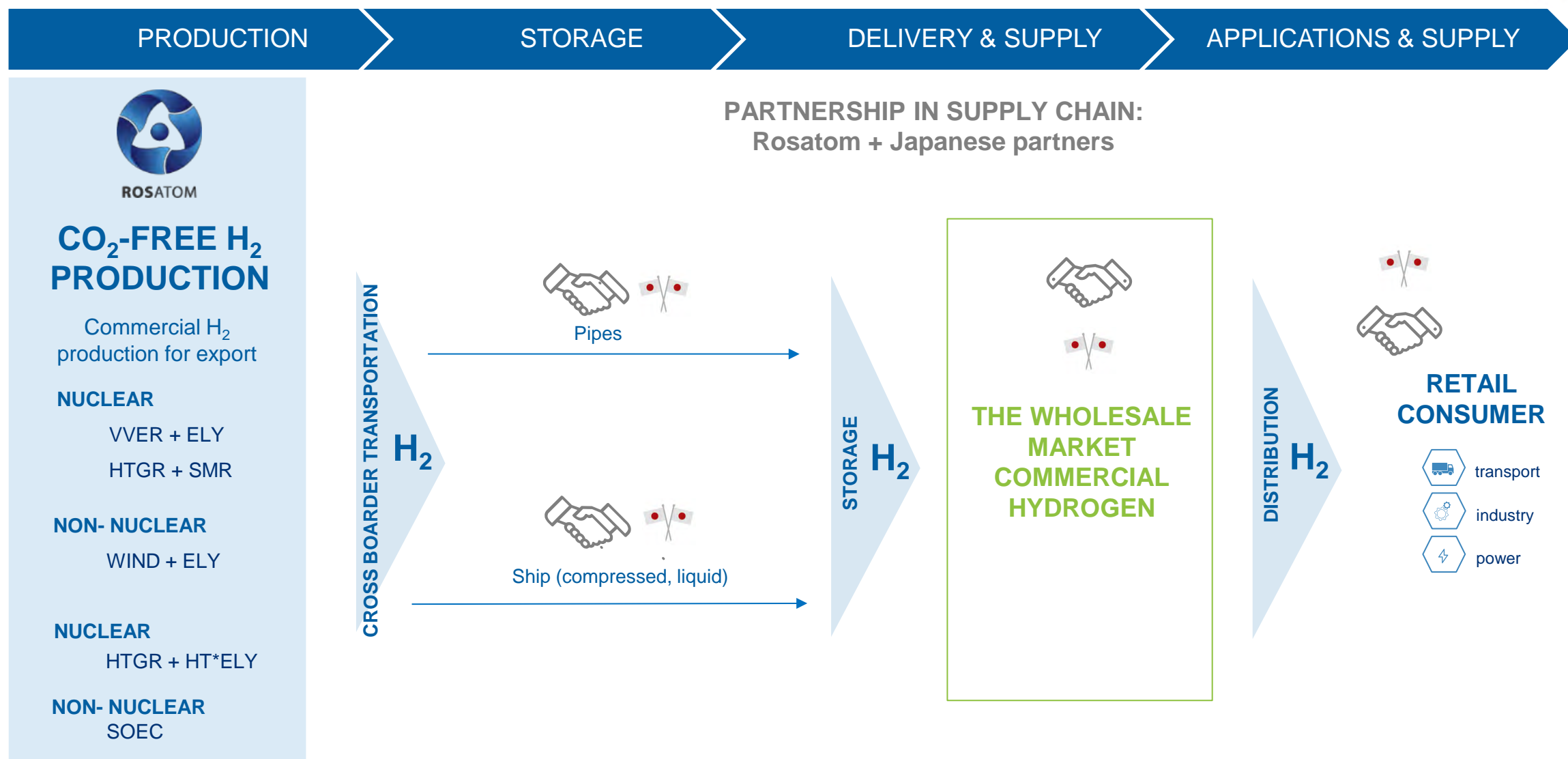
**TRANSITION TO A CO2-FREE TECHNOLOGIES FOR  
THE ENTIRE PRODUCTION CHAIN**

Large scale applications:



**RUSSIA CAN BECOME ONE OF RELIABLE SUPPLY CHAIN PARTNERS TO JAPAN**

# Rosatom is Ready to Cooperate



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